

FORTEVRAUD 11

Contribution of Materials Investigations and Operating Experience to LWRs' Safety, Performance and Reliability

PROGRAM



September 14 to 16,
2026



Palais des Papes,
Avignon

Organizer



Partner



EUROPEAN NUCLEAR SOCIETY



As the tradition goes, the International Symposium Fontevraud 11 will be dedicated to the « **Contribution of Materials Investigations and Operating Experience to LWRs' Safety, Performance and Reliability** ».

This conference is devoted to promote exchanges regarding the feedback provided by materials analyses and investigations for the improvement on the operation and safety of light water nuclear reactors. The topics usually involved in this conference include all experiences gained by operators and research entities in the analysis of field-experienced failures, wear or degradation, detection of defects, components control and any materials research investigation connected to field failures.

In this perspective, the Fontevraud Conference is a strong opportunity for operators and researchers to share detailed gained experience provided in the framework of their respective failures investigations programs. This conference mainly addresses LWRs operating experience from the materials point of view in fields as diverse as surveillance programs, corrosion or fatigue investigations and studies, wear, corrosion and aging management.

Presentations will be organized around key technical issues or systems and components :

Pressure Vessel Components | Pressure Vessel Internals | Stainless Steel & Nickel-based Alloys Areas | Piping, Pumps, Valves | Steam Generators | Steam-Water Systems | Turbine, Alternator | Fuel, Control Rod Assembly | Civil Engineering | Non-metallic Materials and Coatings | FAC Flow Accelerated Corrosion

For the 11th edition of Fontevraud conference, the program committee will particularly welcome papers incorporating **Artificial Intelligence tools in the symposium's theme.**



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SECRETARY : **Mickael SERRIERE**,
Framatome

PROGRAM COMMITTEE

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 **Anne DEMMA**, KAIROS Power

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 **Faiza SEFTA**, OECD-NEA

TOPICS:

T1. Pressure Vessel Components

T2. Pressure Vessel Internal

T3. Stainless Steel & Nickel-based Alloys Areas

T4. Piping, Pumps, Valves

T5. Steam Generator

T6. Steam Water Systems

T7. Turbine, Alternator

T8. Fuel, Control Rod Assembly

T9. Civil Engineering

TB. Flow Accelerated Corrosion

SFEN EVENTS TEAM

sfen.events@sfen.org

P. Hamel-Bloch, Event Director

S. Abbadi, J. Barbier, Event Team

POSTERS:

P3. Stainless Steel & Nickel-based Alloys Areas

P5. Steam Generator

PB. Flow Accelerated Corrosion

PROGRAM OVERVIEW

MONDAY	TUESDAY	WEDNESDAY
Convention center opening Check-in	Convention center opening Check-in	Convention center opening Check-in
Welcome address Opening keynote speeches	Technical sessions T01.2 T03.3 T09	Technical sessions TB
Poster pitches session	Technical sessions T02.2 T03.4 T06	Panel discussion
Welcome reception	Buffet lunch	Buffet lunch
Keynotes	Technical sessions T01.3 T03.5 T04.2	Technical sessions T01.4 + T8 T05.1
Technical sessions T01.1 T03.1+T04.1	Technical sessions T02.3 T03.6 T04.3	Technical sessions T02.4 T05.2
Technical sessions T02.1 T03.2	Conference party	

Technical sessions

- T01.1** Pressure Vessel Components - Irradiation ageing effects
- T01.2** Pressure Vessel Components - Manufacturing and environment effect issues
- T01.3** Pressure Vessel Components - Fracture mechanics
- T01.4** Pressure Vessel Components - Monitoring of irradiation effects
- T02.1** Pressure Vessel Internals - Component management and wear
- T02.2** Pressure Vessel Internals - Mechanical analysis and fracture toughness testing
- T02.3** Pressure Vessel Internals - Evolution of microstructural and mechanical properties under irradiation
- T02.4** Pressure Vessel Internals - Effects of environment and IASCC
- T03.1** Stainless Steel & Nickel-Based Alloys Area - Stress corrosion cracking of stainless steel piping
- T03.2** Stainless Steel & Nickel-Based Alloys Area - Stainless steels 1/3
- T03.3** Stainless Steel & Nickel-Based Alloys Area - Stainless steels 2/3
- T03.4** Stainless Steel & Nickel-Based Alloys Area - Stainless steels 3/3
- T03.5** Stainless Steel & Nickel-Based Alloys Area - Nickel alloys 1/2
- T03.6** Stainless Steel & Nickel-Based Alloys Area - Nickel alloys 2/2
- T04.1** Piping, Pump, Valves - Stress corrosion cracking of stainless steel piping
- T04.2** Piping, Pump, Valves - Experience feedback
- T04.3** Piping, Pump, Valves - Miscellaneous material investigations
- T05.1** Steam Generator - Inconel 690 tube bundle and plugs
- T05.2** Steam Generator - SG secondary side
- T06** Steam Water Systems
- T08** Fuel, Control, Rod assembly
- T09** Civil Engineering
- TB** Flow Accelerated Corrosion



Monday 14 September

08:30 - 09:30

Congress center opening - Check-in

09:30 - 10:15

Welcome Address & Opening Speech

Welcome Address: Valérie FAUDON, Sfen Executive Director - Miguel YESCAS, Fontevraud 11 Program Committee Chair

Opening Speech: Etienne DUTHEIL, Sfen Chairman and Director of Nuclear Operation Division - EDF

10:15 - 11:00

Keynote 1

Materials Investigations on Harvested Components from the Fessenheim NPPs within EDF's LTO program

To support its LTO beyond 60 years objective and materials ageing stakes, EDF considers as an opportunity to harvest components from Fessenheim NPPs which are under dismantling, in addition to domestic or international on-going programs. From Reactor Pressure Vessel to valves, through cables and concrete drilling, the materials and components to be harvested, tested and characterized will extensively reinforce ageing knowledge, support new methodologies and scientific codes robustness, and allow to verify the absence of unforeseen phenomena.

Anne-Françoise COTTE, Project Manager on Fessenheim Harvesting Program and Ageing Management within EDF's LTO Project

11:00 - 11:30

Poster pitches session

List of posters at the end of this daily program

11:45 - 13:15

Welcome reception

13:15 - 13:40

Keynote 2

074 - A Review of Major Topics of Corrosion and Environmentally Assisted Cracking of Steels for Mechanical Components of Light Water Reactors from the Past 40 Years

A. Roth (Consultant, Germany)

13:40 - 14:05

Keynote 3

121 - EDF SCC operating experience on SIS and RHR stainless steels piping: from understanding to mitigation

F. Foct (EDF, France)

14:05 - 14:20

Transition for room changes

14:20 - 15:40

 T01.1 - PRESSURE VESSEL COMPONENTS - Irradiation ageing effects

041 - Microstructural characterization of low-dose neutron irradiated RPV material harvested from a decommissioned Japanese PWR plant

Y. Maeda (Institute of Nuclear Safety System, Incorporated., Japan)

033 - Update of Microstructure Characterization of RPV Material Harvested from a Japanese PWR Plant

K. Fujii (Institute of Nuclear Safety System, Inc., Japan)

150 - High Resolution Microstructural Characterization of Nanoscale "Features" in Neutron-Irradiated RPV Materials: Applications to High Ni-Mn Welds and A508 Gr3 Steel

M.G. Burke (Idaho National Laboratory, United States of America)

147 - Restoration of Irradiation Damage in Reactor Vessel Steels: Insights from Atomic-Level Characterization

C. Hossepied (CEA, France)

14:20 - 15:40

 T03.1+T04.1 - STAINLESS STEEL & NICKEL-BASED ALLOYS AREAS + PIPING, PUMPS, VALVES - Stress corrosion cracking of stainless steel piping

107 - Residual Stress and Strain in Austenitic Stainless-Steel Welds: Integrated Approach for PWSCC Risk Assessment

I. Bahtit (EDF, France)

130 - Surge line welds SCC susceptibility assessment

F. Foct (EDF, France)

122 - Weld root grinding benefits regarding SCC risk – operating experience and mock-ups characterization

I. Bahtit (EDF, France)

105 - SEM-FIB and TEM investigations of the evolution of the oxide and microstructure in base metal at weld roots from EDF auxiliary lines

P. Cuvillier (EDF, France)

15:40 - 16:20

Coffee break

16:20 - 18:00

 T02.1 - PRESSURE VESSEL INTERNALS - Component management and wear

005 - An Update for Aging Management of PWR Internals in U.S. Plants

K. Amberge (Electric Power Research Institute, United States of America)

049 - Laboratory test evaluation of the wear kinetics of thermal sleeves

M. Mori (Mitsubishi Heavy Industries, Japan)

111 - Investigation of wear mechanisms of thermal sleeves through examinations of decommissioned sleeves and laboratory wear tests

R. Bonzom (EDF R&D, France)

110 - Thermal Sleeve Flange wear: analysis of FROG member operational experience
R. El Hourany (Framatome, France)

129 - Tihange 3 – Guide Tubes Wear mitigations
S. De Vroey (Engie Laborelec, Belgium)

16:20 - 18:00

 T03.2 - STAINLESS STEEL & NICKEL-BASED ALLOYS AREAS - Stainless steels 1/3

035 - Local microstructural characterization of small-strain hardening in large austenitic stainless-steel components
E. Plancher (Framatome DTIM - Mechanical Engineering, France)

042 - Structural impacts of viscoplastic flow during hyperquenching of large components in austenitic stainless steel
R. Lacroix (Framatome, France)

059 - Updated Evaluation of the Effect of Sulfur on SCC Growth of Stainless Steel in Hydrogenated Water
T. Moss (Naval Nuclear Laboratory, United States of America)

071 - Update of initiation and crack growth models for a better prediction of SCC of cold worked stainless steels
T. Couvant (EDF, France)

053 - Comparison of Fatigue Crack Mechanisms of 304L Austenitic Stainless Steel in Air and in PWR Primary Environment
N. Huin (Canadian Nuclear Laboratories, France)

18:00

End of day 1

 Tuesday 15 September

08:30 - 09:00

Congress center opening - Check-in

09:00 - 10:20

 T01.2 - PRESSURE VESSEL COMPONENTS - Manufacturing and environment effect issues

052 - Fracture behavior of low-alloy RPV steels in the ductile–brittle transition region: influence of processing and test conditions
I. Uytendhouwen (SCK CEN, Belgium)

088 - Effect of the intercritical heat treatment on the microstructure and impact properties of an MnNiMo bainitic steel
F. TIOGUEM (FRAMATOME Le Creusot, France)

093 - Contribution of local segregations to the scatter of Charpy impact test results in PWR Pressure Vessel Steels, and its analysis with a numerical model of microstructure generated by AI
P. Joly (Framatome, France)

019 - Environmentally-assisted cracking of a thermally-aged and irradi-ated RPV steel in oxygenated high-temperature water
H.P. Seifert (Paul Scherrer Institut, Switzerland)

09:00 - 10:20

 T03.3 - STAINLESS STEEL & NICKEL-BASED ALLOYS AREAS - Stainless steels 2/3

025 - Oxygen Effects on Stress Corrosion Cracking Initiation of Stain-less Steel in PWR Primary Water

Z. Zhai (Pacific Northwest National Laboratory, United States of America)

099 - Ongoing research program investigating environmental effects in simulated PWR primary water on Stress Corrosion Cracking susceptibility of Cold Worked Austenitic Stainless Steels

C. Rainasse (EDF, France)

128 - Mitigating Stress Corrosion Cracking Initiation in Cold-Worked 316L Stainless Steel under PWR Conditions via Zinc Injection

T. Wang (Paul Scherrer Institute, Switzerland)

047 - Development of a generic intergranular oxidation model of Fe-Cr-Ni alloys in PWR primary water

L. Fasquel (EDF, France)

09:00 - 10:20

 T09 - CIVIL ENGINEERING

067 - Development of a Quantitative Evaluation Method for Carbonation Depth in Concrete Structures of Nuclear Power Plants Using an Electrochemical Method

M. Maeda (Central Research Institute of Electric Power Industry, Japan)

065 - Investigation of MIC Mechanisms in Gabion Structures and the Role of Cathodic Protection

C. Loutfi (EDF, France)

154 - Thermo-hydro-mechanical Study of Concrete Creep at 150 °C and 90% HR: Separating Basic and Drying Creep for Improved Parameter Calibration

S. CHENG (CEA Paris Saclay, France)

142 - Study of the aging of an epoxy coating in the reactor building of a nuclear power plant under accident conditions

A. Dumand (EDF Direction Technique TEGG, France)

10:20 - 10:50

Coffee break

10:50 - 12:30

 T02.2 - PRESSURE VESSEL INTERNALS - Mechanical analysis and fracture toughness testing

139 - Irradiation Embrittlement of Reactor Pressure Vessel Internals – Testing of Materials Harvested from Decommissioned Swedish Reactors

J. Stjärnsäter (Studsvik Nuclear AB, Sweden)

160 - Ductile Instabilities and Effect on Irradiated Fracture Toughness

B. Hall (Westinghouse, United States of America)

124 - Room temperature fracture properties of a SA304 austenitic stainless steel irradiated in LWR conditions

J. Hure (CEA, France)

031 - Lessons learned from numerical interpretations of toughness tests performed on highly degraded irradiated stainless steels

S. Chapuliot (EDF Lab Renardières, France)

104 - Flaw evaluation of PWR core barrel in Japan

K. Tahara (Mitsubishi Heavy Industries, Japan)

10:50 - 12:30

● **T03.4 - STAINLESS STEEL & NICKEL-BASED ALLOYS AREAS - Stainless steels 3/3**

036 - Tensile behaviour of Z10C13 martensitic stainless steel under dynamic loading at intermediate strain rates and high temperature

E. Plancher (Framatome DTIM - Mechanical Engineering, France)

127 - Collaborative Study on the SCC Susceptibility of Additively Manufactured Stainless Steels in Light Water Reactor Environments: First Findings of the CAISAM Project

T. Babinský (PSI Center for Nuclear Engineering and Sciences, Switzerland)

152 - Stress corrosion cracking of additive manufactured 316L stainless steel in pressurized water reactor primary water

J. Nguejio (CEA, France)

118 - Mitigation of SCC in CANDU Reactor Calandria Relief Ducts

J. Smith (EPRI, Canada)

075 - International activity on comparison of stress corrosion cracking model predictions with operating experience, Part 1: Description of Methodology

S. Faust (GRS, Germany)

10:50 - 12:30

● **T06 - STEAM WATER SYSTEMS**

081 - Management of Erosive Degradation Mechanisms in Nuclear Power Plants

R. Wolfe (EPRI, United States of America)

125 - Cavitation Awareness in Operation and Design

M. Weiss (Framatome GmbH, Germany)

126 - Degradations on the APG001RF heat exchangers of 4 NPP units - Impact analysis of the chemistry in operating cycle and layup practices

S. Lamoudi (EDF, France)

114 - Degradations affecting metal bellows expansion joints of low pressure heat exchanger steam feed lines

J.F. Coste (ELECTRICITE DE FRANCE, France)

092 - Impact of the Nb and V additions on the hardness in the HAZ of a microalloyed carbon-manganese steel and its fracture toughness

G. Ben Salem (Framatome, France)

12:30 - 14:00

Buffet lunch

14:00 - 15:20

● **T01.3 - PRESSURE VESSEL COMPONENTS - Fracture mechanics**

086 - Barsebäck Reactor Pressure Vessel Toughness Variation Through Wall Thickness

S. Lindqvist (VTT, Finland)

148 - Application of miniature-C(T) for determination of the master curve of a highly irradiated 16MND5 RPV steel

S. Saanouni (CEA, France)

149 - Local approach to fracture-based assessment of the Master Curve reference temperature T₀ from mini-C(T) geometry of a highly irradiated RPV steel
P. Francois (CEA, France)

007 - Investigation of the constraint effect on irradiated RPV steels by testing shallow crack SE(B) specimens
J. May (Framatome GmbH, Germany)

14:00 - 16:00

 T03.5 - STAINLESS STEEL & NICKEL-BASED ALLOYS AREAS - Nickel alloys 1/2

027 - Degradation of Forsmark 3 bottom nozzle attachment welds
F. Gustavsson (Forsmark Nuclear Power Plant, Sweden)

078 - International activity on comparison of stress corrosion model predictions with operating experience, part 2: Crack growth in Ni-based welds
S. Faust (GRS, Germany)

064 - Evaluating the efficacy of cavitation peening for SCC resistance in Alloy 182 under simulated light water reactor conditions
A. Das (Paul Scherrer Institut, Switzerland)

043 - Effect of Operation and Thermal Aging on Microstructure, Mechanical Behavior and Cracking of A52 Dissimilar Metal Welds
S. Lindqvist (VTT, Finland)

034 - Stress Corrosion Cracking Growth Rates in Additively Manufactured Laser-Powder Bed Fusion Alloy 625
A. Pinkowitz (Knolls Atomic Power Laboratory, United States of America)

108 - Preliminary FOI's evaluation of primary pump studs replacing A286 stainless steel by Alloy 718
Q. Tence (EDF DISC DT, France)

14:00 - 15:20

 T04.2 - PIPING, PUMPS, VALVES - Experience feedback

089 - Fracture toughness of a SA335 P1 steel from the original Feed water Pipe Line part of a NPP
I. Uytendhouwen (SCK CEN, Belgium)

131 - Thermal fatigue - Characterization of sections of Reactor Heat Re-moval System from service in 900 and 1300 MW plants: review of failure investigations since 2011
F. Renaud (EDF - Technical Direction, France)

006 - Thermal Cycling in Normally Stagnant RCS Branch Lines connected to the Residual Heat Removal Systems of the Doel 4 and Tihange 3 Nuclear Power Plants
M. De Smet (TRACTEBEL-ENGIE, Belgium)

113 - Failure Analysis of Krško SI-53 Piping
S. El Shawish (Jožef Stefan Institute, Slovenia)

16:00 - 16:30

Coffee break

16:30 - 17:50

T02.3 - PRESSURE VESSEL INTERNALS - Evolution of microstructural and mechanical properties under irradiation

146 - Microstructural Evolution of Irradiated 403 Stainless Steel CANDU End Fittings

R. Matthews (Canadian Nuclear Laborat, Canada)

133 - Effect of Irradiation on the NRU G-16 Fast Neutron Cup Under CANDU-Relevant Conditions – Helium Behaviour and Microstructural Evolution

M. Mattucci (Canadian Nuclear Laboratories , Canada)

135 - Sensitivity analysis of swelling in irradiated austenitic stainless steels: a comparison between experimental data and cluster dynamics modeling

G. Adjanor (EDF R&D, France)

022 - Micro-tensile testing of grain boundaries in Fe and He ion-irradiated type 316 stainless steel and model alloys

T. Miura (Institute of Nuclear Safety System, Inc., Japan)

16:30 - 17:50

T03.6 - STAINLESS STEEL & NICKEL-BASED ALLOYS AREAS - Nickel alloys 2/2

021 - Improving SCC prediction of Bottom Mounted Instrumental nozzles

C. Perez (EDF Lab Renardières, France)

062 - Improvement of Alloy 600 SCC initiation and crack growth models to better evaluate to susceptibility to SCC of components

T. Couvant (EDF, France)

094 - Understanding Long-Term Crack Initiation Behavior in Cold Worked Alloy 690

Z. Zhai (Pacific Northwest National Laboratory, United States of America)

096 - Influence of cold work on mechanical behavior of Alloy 690

J. Blaizot (Framatome, France)

16:30 - 17:30

T04.3 - PIPING, PUMPS, VALVES - Miscellaneous material investigations

134 - Buried Piping Non-Metallic Rehabilitation Technologies for the Refurbishment of Nuclear Stations

C. Cooper (Kinectrics Inc, Canada)

029 - Evaluation of Gas Metal Arc-Direct Energy Deposition Type 316LSi Stainless Steel Valves

F. Gift (Electric Power Research Institute (EPRI), United States of America)

085 - Mechanical testing of thermally aged stainless steel – Base and Welding materials

R. Magnusson (Ringhals - Vattenfall, Sweden)

17:50

End of day 2

19:30 - 22:00

Conference party

Wednesday 16 September

08:15 - 08:45

Congress center opening - Check-in

08:45 - 10:45

TB - FLOW ACCELERATED CORROSION

057 - Examination of an expander affected by FAC on the secondary circuit, located downstream of a weld and after a regulating valve

P. Mestre-Rinn (EDF, France)

008 - Identification of local flow-accelerated corrosion phenomena using data analysis methods

V. Garric (EDF, Direction Technique, France)

158 - Massive audit on XRF trace chrome measurements for FAC mitigation

D. Delacoux (EDF DTG, France)

044 - CFD simulations of flow accelerated corrosion coupled with electrochemistry

S. Mimouni (EDF Lab Chatou, France)

045 - Computational Fluid Dynamics at the Entrance of Stagnant Lines for Flow-Accelerated Corrosion

J. Varnam (EPRI, United States of America)

032 - Predicting Flow-Accelerated Corrosion and Erosion in Nuclear Power Plants the Homogeneous and Slip Models for Two-Phase Steam Flow

R. Wolfe (EPRI, United States of America)

10:45 - 11:45

Panel discussion

Artificial Intelligence - Legitimate caution or unjustified delay ?

The safety and performance of nuclear power plants largely depend on the long-term behavior of materials. These materials are subjected to multiple stresses: neutron irradiation, corrosion, thermal aging, fatigue, operational transients, which generate complex and often coupled damage mechanisms.

The Fontevraud conference covers all these themes as well as the Operational Experience (OPEX) accumulated over several decades. In this context, Artificial Intelligence opens new perspectives today to support operators and materials experts. AI is currently at the forefront of technological discussions; however, its application is not straightforward, due to ambitious objectives, and difficulties related to available data.

Nevertheless, several countries have already made significant investments in this field and are beginning to obtain promising results, both in predictive maintenance and in the understanding of degradation mechanisms. In this rapidly evolving landscape, several key questions emerge. This panel will explore some of these questions and discuss potential paths forward.

Moderator: **Christophe DOMAIN**, EDF R&D

Panelists: **Gareth HOPKIN**, Nuclear Inspector and Non-Executive Director, ONR - **Franck TANCRET**, Professor/Researcher, Polytech, Nantes Université, **Brent CAPELL**, Principal Team Leader, EPRI - **Quentin MISTRAL**, NDT an IA Expert, EDF

11:45 - 12:45

Buffet lunch

12:45 - 14:05

 T01.4 - PRESSURE VESSEL COMPONENTS - Monitoring of irradiation effects

145 - Characterization of High-Fluence RPV Surveillance Capsule from Palisades NPP

M. Sokolov (ORNL, United States of America)

157 - A Deep Learning Framework for Automated Detection of Intergranular Rupture in SEM Fractographs of RPV Steel

A. Courcelle (CEA, France)

 T08 - FUEL, CONTROL ROD ASSEMBLY

083 - Mechanistic study of the oxidation behavior of Zr-1Sn-1Nb-0.3Fe alloy in simulated PWR primary water

Z. Li (Institute of Corrosion Science and Technology, China)

051 - Release of fission products from a failed rod and influence of the oxidation state of its pellets

C. Petit (Framatome, France)

12:45 - 14:45

 T05.1 - STEAM GENERATOR - Inconel 690 tube bundle and plugs

102 - High-temperature aqueous corrosion studies of Alloy 690 steam generator tubes

M. Zimina (University of Bristol, School of Physics, HH Wills Physics Laboratory, United Kingdom)

143 - Alloy 690TT Production Method SCC Resistance Testing

B. Capell (EPRI, United States of America)

137 - SMILE Project: Characterization on Harvested Ringhals 2 Alloy 690 TT Steam Generator Tubes

M. Wang (Studsvik Nuclear AB, Sweden)

028 - Sherlock Project – Investigate to Mitigate

F. Foct (EDF, France)

058 - Examination of an Inconel 690TT mechanical steam generator plug after 20 years of operation

P. Mestre-Rinn (EDF, France)

153 - Eddy Current Simulation for Steam Generator Tube Inspections

J. Benson (EPRI, United States of America)

14:45 - 14:55

Transition

14:55 - 16:15

 T02.4 - PRESSURE VESSEL INTERNALS - Effects of environment and IASCC

103 - TEM investigations of oxides and base metal at the surface of the confined zone of an uncracked baffle-former bolt irradiated up to 40 dpa coming from a French PWR

P. Cuvillier (EDF, France)

123 - Quantification of grain-scale localized plasticity by micro-DIC for developing physical models of IASCC

W. Karlsen (VTT Technical Research Centre of Finland, Finland)

004 - Hot Cell Testing of Type 304 Stainless Steel Core Barrel Samples Removed from U.S. PWR

F. Gift (Electric Power Research Institute (EPRI), United States of America)

076 - Effects of Chloride and Sulfate on the Crack Growth Rate of Irradiated Stainless Steel in BWR Environments

J. Stjärnsäter (Studsvik Nuclear AB, Sweden)

14:55 - 16:15

 T05.2 - STEAM GENERATOR - SG secondary side

009 - Understanding the spatial distribution of clogging inside steam generators by data analysis

V. Garric (EDF, Direction Technique, France)

084 - Development of simulation method for boiling two-phase flow in the secondary side of steam generators

J. Suzuki (Mitsubishi Heavy Industries, Kobe Shipyard, Japan)

098 - The concept of SCC precursors, experimental work, and its application to Canadian SG tube extracted from Periodic Inspection Programs

N. Huin (Canadian Nuclear Laboratories, France)

079 - Decrease of hydrogen content in low alloyed steels for steam generator parts

A. Guenot (Framatome Le Creusot, France)

16:15

End of the conference

Posters

P1 - PRESSURE VESSEL COMPONENTS

132 - Evolution of Irradiation-Induced Microstructure in Reactor Pressure Vessel Materials for Long-Term Operation
T. Esclapez (EDF Lab Renardières, France)

P3 - STAINLESS STEEL & NICKEL-BASED ALLOYS AREAS

046 - Development of the Ultra-micro Tensile test system for Crack Tip (UTCT)
K. Fujii (Institute of Nuclear Safety System, Inc., Japan)

116 - Impact of Cleanliness of Austenitic Stainless Steel 304 on Corrosion and Cracking Susceptibility After Long-Term Service in Nuclear Environments
J. Ben Mohamed (Framatome, France)

138 - Effect of Zn injection on high-temperature water oxidation behaviour in LWR steam generator materials
B. Carley-Macaulay (University of Bristol, United Kingdom)

106 - Evaluation of the PWSCC susceptibility of nickel base alloys HAZ using calibrated welding defects
J. Caballero-Hinostroza (Framatome, France)

P9 - CIVIL ENGINEERING

159 - Neutron flux effect on concrete shield building from meso-scale to structure-scale
K. Hrdličková (Research Centre Řež, Czech Republic)

PB - FLOW ACCELERATED CORROSION

039 - Examination of steam generator swirl vanes affected by FAC
S. Le Ber (EDF, France)

Useful information

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Avignon Tourisme - 6, rue Pente Rapide

Charles Ansidei BP149 - 84008 AVIGNON



Plan your stay

How to come?

BY PLANE

Marseille Provence Airport

45 mins drive from Avignon

110+ direct destinations in 29 countries

BY TRAIN

2 STATIONS

- Avignon Center Station

The venue is 15mn walking from the Center Station

- Avignon TGV Station

Regular TER trains connect Avignon TGV Station to Avignon Centre Station in approximately 4-5 minutes.

BY TAXI

Taxis Radio Avignonnais

+ 33 (0)4 90 82 20 20

BY CAR

- A7 motorway

Avignon Sud / Avignon Nord

- A9 motorway

Remoulins exit

- RN7 and RN100

Accommodation

Due to a peak traffic period, we highly recommend you to book your transport tickets and accommodation as soon as possible.



• [Novotel Avignon Centre](#)

• [Avignon Grand Hôtel](#)

• [Hôtel Cloître Saint Louis](#)

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• [Ibis Avignon Centre Gare](#)

• [Ibis Avignon Centre de l'Europe](#)



• [Ibis budget Avignon Centre](#)

Restaurants

Below, you will find a range of local restaurants within a short distance of the venue. There's something for everyone!



FOR A QUICK BITE TO EAT

- **LA BRASSERIE DE L'HORLOGE**
26 Place de l'Horloge, 84000 Avignon
+33 7 72 77 87 56
- **PINOCCHIO**
29 Rue Théodore Aubanel, 84000 Avignon
+33 4 90 82 60 90



FRENCH TRADITIONNAL CUISINE

- **LE MOUTARDIER DU PAPE**
15 Place du Palais 84000 Avignon
+33 (0)4 90 85 34 76
- **LA SALLE À MANGER**
4 place de l'Amirande 84000 Avignon
+33 (0)4 90 14 20 20
- **LE CARRÉ DU PALAIS**
1 Place du Palais 84000 Avignon
+33 (0)4 65 00 01 01



FOR SPECIAL OCCASIONS BUT ALSO AFTER HARD DAYS WORK *RESERVATION REQUIRED*

- **POLLEN**
18 rue Joseph Vernet, 84000 Avignon
+33 (0)4 86 34 93 74
- **LA MIRANDE**
4 place de l'Amirande, 84000 Avignon
+33 (0)4 90 14 20 20



MICHELIN

2026

FRANCE



<https://www.sfen.org/evenement/fontevraud-11/>

Organizer



Partner

