



GLOBAL 2022

INTERNATIONAL CONFERENCE ON NUCLEAR FUEL CYCLE NEW NUCLEAR PERSPECTIVES IN THE ENERGY SUPPLY CRISIS AND CLIMATE EMERGENCY

 July 6 to 8, 2022

 Reims, France

PROGRAM

Get ready to network !

ORGANIZED BY



IN PARTNERSHIP WITH



PLATINUM SPONSORS



SILVER SPONSOR



Updated July 04, 2022

WEDNESDAY 6	THURSDAY 7	FRIDAY 8
8:30 Congress Center Opening	8:00 Congress Center Opening	8:00 Congress Center Opening
9:30 Welcome Address & Opening - Plenary 1	8:30 Technical sessions in // 3b* 5b 6a	8:30 Technical sessions in // 1a+1b 2c 3a** 6c
11:00 Plenary 2	10:30 Break	10:30 Break
12:30 Exhibition opening & Welcome Cocktail	11:00 Plenary 3	11:00 Plenary 4
2:00 Technical sessions in // 2a 3c 5a	12:30 Lunch	12:30 Closing
4:00 Break	2:00 Technical sessions in // 4a+4b 5d 6b	12:45 Lunch
4:30 Technical sessions in // 3d 5c 2d	4:00 Break	1:30 End
6:30 End of day 1	4:30 Technical sessions in // 2b 5e 6b	----
	6:45 Keynote speech	Technical visit(a)
	7:30 Conference reception	▪ 12:30 pm: Departure from Reims Congress Center
	-----	▪ 2:30 pm: Visit
	*Special session 1 Nuclear mining techniques and Sustainability	▪ 4:30 pm: Departure for Troyes Train Station
	-----	-----
		**Special session 2 Front ends markets: updates and prospects

1 • Nuclear energy outlook and prospects

1a • Nuclear energy in the energy transition | 1b • Impact of current trends on the nuclear cycle

2 • Fuel Cycle Options including technical-economical evaluations

2a • Industrial practices in water-cooled reactors | 2b • Advanced fuel cycle options for thermal neutron reactor systems | 2c • Focus on MSRs as a fuel cycle option 9 | 2d • Other Generation IV nuclear systems as a fuel cycle option

3 • Nuclear fuel cycle front-end

3a • Front ends markets: updates and prospects | 3b • Nuclear mining techniques and Sustainability | 3c • Uranium processing and conversion and enrichment | 3d • Nuclear fuel design and fabrication: includes innovations, ATF

4 • Spent fuel storage & transportation

4a • Extension of storage period: ageing management, relicensing.. | 4b • Long term storage systems and behavior of SF under storage, SF integrity under storage conditions and innovation

5 • Fuel recycling

5a • LWR process head and extraction | 5b • LWR Fission products | 5c • LWR Analysis | 5d • LWR Mixed oxide fuel manufacturing and reprocessing – Fast neutron reactor fuel manufacturing and reprocessing (MOX, metal fuel) | 5e • Partitioning and transmutation: fuels and recycling processes

6 • Waste management

6a • Waste management Strategy / Options | 6b • Waste Treatment / Conditioning | 6c • Disposal / Storage

	6	7	8		6	7	8
NUCLEAR ENERGY OUTLOOK AND PROSPECTS		X		Plenaries		X	X
FUEL CYCLE OPTIONS INCLUDING TECHNICAL-ECONOMICAL EVALUATIONS	X	X	X	Technical sessions in //		X	X
NUCLEAR FUEL CYCLE FRONT-END	X	X	X	Technical Visit			X
SPENT FUEL STORAGE & TRANSPORTATION		X		Posters		X	X
FUEL RECYCLING	X	X					
WASTE MANAGEMENT	X	X					

JULY, WEDNESDAY 6

8.30	Registration • Congress Centre Main Entrance	
9.30-11.00	Royale Room	Welcome & Opening Plenary 1 Welcome address and opening • P. Stohr (CEA), G. Dureau (Orano)
	Royale Room	Plenary 1 Nuclear power contribution to support the security of energy supply of countries weakened by their crude oil and natural gas import dependency while contributing to their climate-neutrality ambition, with a focus on impact on the nuclear fuel cycle > Moderator : G. Dureau (Orano) R. Grossi (AIEA), S. Arai (JAIF), R. Arnold (BEIS), D. Cameron (OECD-NEA), A. Griffith (DOE), M. Hübel (EC), S. Mourlon (DGEC), S. Wang (CNS)
11.00-12.30	Royale Room	Plenary 2 Fuel cycle options considering circular economy, environmental impact, and public support to enhance the positioning of nuclear power within the low CO2 energy mix > Moderator : P. Stohr (CEA) H. Mizuta (Kansai EPCO), R. Buisset (EDF), A. Gay (Orano), A. Griffith (DOE), D. Mathers (NIRO)
12.30-14.00	Exhibition Opening & Welcome cocktail Sponsored by CEA	
14.00-16.00	Diderot	2a - FUEL CYCLE OPTIONS INCLUDING TECHNICAL-ECONOMICAL EVALUATIONS - Industrial practices in water-cooled reactors > Chairs: G. Grassi (OECD-NEA), I. Morlaes (Orano)
		223 • Integrated approach in nuclear spent fuel and waste management D. Hambley (UK NNL on behalf of WNA SUFM WG)
		218 • IAEA On-going Activities on Power Reactor Fuels and Fuel Cycle Options for Nuclear Energy Sustainability B. Moldovan (IAEA)
		19 • Advanced modeling of the historical French fleet with DON-JON5/CLASS coupled simulations X. Doligez (CNRS)
		61 • Study of nuclear fleets in a steady-state regime using the tool SEPAR H. Barale (CEA)
		165 • Cost/Risk-mitigating back-end fuel cycle strategies: reconciling vision with economics L. Van Den Durpel (Nuclear-21)
14.00-16.00	Rimbaud	3c - NUCLEAR FUEL CYCLE FRONT-END - Uranium processing and conversion and enrichment > Chair: N. Vioujard (Framatome)
		151 • Biohydrometallurgy for Uranium Dissolution – IRONBIOX, SOMAIR Pilot = Innovating for Enhanced Performance C. Leycuras (Orano)
		153 • Zeolite, a promising sustainable treatment for radium removal from mining water J. Schick (Orano)
		90 • Toward a fine knowledge of mass transfer kinetics, contribution from microfluidic devices A. Lélias (CEA)
		94 • Industry 4.0 transformation in the uranium conversion and enrichment sector K. Longuet de la Giraudière (Orano), Z. Mekhalfa (Orano)
14.00-16.00	Colbert	5a - FUEL RECYCLING - LWR process head and extraction > Chair: A. Salvatores (CEA)
		8 • Exploring Options for Recycling of Uranium from HALEU-Based Fuel G. Lumetta (Pacific Northwest National Laboratory)
		192 • Preparation and Small Scale Dissolution of Homogenous (U,Pu) Mixed Oxides R. Sanderson (National Nuclear Laboratory)
		99 • Dissolution phenomena of mixed (U,Pu) oxides G. Garzon Losik (CEA)
		63 • Demonstration trials using lab-scale pulsed columns of a new spent nuclear fuel treatment process H. Roussel (CEA)
		109 • Experimental and modeling study of uranium(VI) extraction with a N,N-dialkylamide solvent P. Moeyaert (CEA)
16.00-16.30	Coffee break	
16.30-18.30	Diderot	3d - NUCLEAR FUEL CYCLE FRONT-END - Nuclear fuel design and fabrication: includes innovations, ATF > Chairs: M. Moatti (EDF), A. Jasulevicius (TVO)
		132 • Response of FeCrAl Alloys to Light Water Reactor Conditions A. Hoffman (GE)
		83 • Characterization of FeCrAl Cladding Tubes in the Entire Fuel Cycle A. Hoffman (GE)
		227 • Protect : the E-ATF solution by Framatome N. Vioujard (Framatome)
		154 • Optimization of core management and fuel assemblies design for MOX recycling in EDF 1300 MW reactors C. Collignon (EDF)
		178 • Micro-scale thermal, mechanical, and residual stress measurements of TRISO particle coatings and 3D-printed fuels A. Leide (University of Bristol)
		148 • Interpretation of thermal conductivity measurements on irradiated (U,Pu)O ₂ fuels for fast reactors (NESTOR experience in the Phenix reactor)" P. Bonev (CEA)
16.30-18.30	Rimbaud	5c - FUEL RECYCLING - LWR Analyse > Chair: J.-M. Marin (Orano)
		67 • Enabling Microscale Processing: Combined Raman and Absorbance Spectroscopy for Microfluidic On-Line Monitoring S. Bryan (Pacific Northwest National Laboratory)
		25 • Microfluidic in-situ spectrophotometric approaches to tackle actinides analysis in multiple oxidation states F. Lamadie (CEA)
		49 • Effect of Third Phase Formation on Microfluidic Extraction A. Brandt (Tokyo Institute of Technology)

⌚ 16.30-18.30 🎪 Colbert

2d - FUEL CYCLE OPTIONS INCLUDING TECHNICAL-ECONOMICAL EVALUATIONS - Other Generation IV nuclear systems as a fuel cycle option

> Chairs: V. Vandenberghe (CEA), B. Moldovan (IAEA)	
125 • An Overview of Activities of the NEA Working Party on Scientific Issues of Advanced Fuel Cycles (WPFC)	G. Grassi (NEA/OECD)
216 • Partitioning of americium and short-lived fission products: towards optimal management of spent nuclear fuel in Belgium	K. Verguts (SCK CEN)
22 • Compatibility between safety requirements and fuel cycle : the case of a small CADOR core	C. Laguerre (CEA)
106 • Physics Study of an Ultra-Long-Life MMLFR (Micro Modular Lead-cooled Fast Reactor) with Low Enriched Uranium Nitride Fuels	Y. Cho (Hanyang University)
50 • Development of highly flexible technology for recovery and transmutation of minor actinides using metal fuel fast reactor	M. Iizuka (Central Research Institute of Electric Power Industry)
162 • 20 years after the OECD/NEA's comparative study on FR and ADS : what have we learned and where do we stand finally ?	L. Van Den Durpel (Nuclear-21)
35 • Advanced nuclear reactors: what about the back-end?	J. Liberge (Orano)

⌚ 18.30 End of day 1

JULY, THURSDAY 7

⌚ 8.00 🏠 Registration • Congress Center Main Entrance

⌚ 8.30-10.30 🎪 Royale Room 3b - NUCLEAR FUEL CYCLE FRONT-END - New mining techniques and Sustainability

SP1	> Chair: T. Gitzel (Cameco)
	221 • Unlock New Resources through Innovation
	171 • Global Atomic : The feasibility study and the development of our Dasa uranium project in Niger
	219 • A new standard of sustainable uranium mining rises from the depths of the uranium down-turn
	220 • ISR Operations in China and the Rossing Uranium Mine in Namibia
	143 • Nuclear system sustainability - Cominak's site remediation

⌚ 8.30-10.30 🎪 Diderot	5b - FUEL RECYCLING - LWR Fission products
	> Chair: G. Grassi (OECD-NEA)
	86 • Material investigation of a dismantled evaporator in Orano La Hague
	96 • Study on effects of composition and heating rate on ruthenium volatilization during decomposition of nitrate mixtures
	111 • Volatilization, transport and mitigation of ruthenium under a loss of cooling accident on high level liquid waste storage tanks in reprocessing plants
	16 • Abatement of iodine from advanced nuclear fuel cycles: A UK perspective
	80 • Abatement of Volatile Organic Iodides from UNF Reprocessing Off-gas

⌚ 8.30-10.30 🎪 Rimbaud	6a - WASTE MANAGEMENT - Strategy / Options
	> Chair: V. Wasselin (ANDRA)
	198 • Options for highly active liquid waste immobilisation for a future UK aqueous recycling plant
	152 • Orano Capabilities in Waste Management
	101 • A new R&D program for disposal of French bitumen radioactive waste in deep geological repository
	121 • Safe Clearance and Recycling of Metals to the Conventional Industry

⌚ 10.30-11.00 🍵 Coffee break	
⌚ 11.00-12.30 🎪 Royale Room	Plenary 3
	> Moderator: D. Delort (ANDRA)

Progress status on DGR development and associated predisposal management for spent fuel and HLW
M. Van Geet (ONDRAF), B. Watts (NWMO), P.-M. Abadie (ANDRA), S. Kondo (NUMO), M. Holmqvist (SKB), K. Wheeler (Nuclear Waste Services Limited), K. Petry (DOE)

⌚ 12.30-14.00 🎪 Lunch

⌚ 14.00-16.00 🎪 Rimbaud	4a, 4b - SPENT FUEL STORAGE AND TRANSPORTATION
	> Chairs: D. Hambley (UK NNL), A. Ambard (EDF)
	77 • Verification of Legacy Methodology of Minimum Critical Mass Estimation at PSI
	225 • An update of the spent fuel research program of the French tripartite institute
	159 • Structural Integrity of Spent Fuel after Long-Term Dry Interim Storage
	118 • Extending the Validation Range for Decay Heat Measurements

⌚ 14.00-16.00

II Colbert

5d - FUEL RECYCLING - LWR ANALYSIS - LWR Mixed oxide fuel and fast neutron reactor fuel manufacturing and reprocessing

> Chair: J.-M. Marin (Orano)

52 • The Process Design of Advanced Nuclear Fuel Fabrication Method	K. Kawauchi (IAEA)
70 • Improvement of density disparity of MOX pellets by modification of depleted UO ₂ powder	Y. Matsumoto (Japan Nuclear Fuel Limited)
129 • Cryo-milling process for MOX nuclear fuel	S. Vaudez (CEA)
87 Manufacturing of (U,Pu)O _{2-x} mixed oxides with high plutonium contents (> 60 mol.%)	M.-M. Desagulier (CEA)
187 • Optimisation of a Flash Sintering Route for UO ₂ and (U,Ce)O ₂ Pellet Fabrication	J. Morgan (The University of Manchester)
74 • Self-irradiation effects on the structural properties of MOx fuels	O. Kahraman (CEA)

⌚ 14.00-16.00

II Diderot

6b - WASTE MANAGEMENT - Treatment / Conditioning

> Chair: F. Hourcade (Orano)

53 • Harmless treatment of radioactive liquid wastes for safe storage in systematic treatment of radioactive liquid waste for decommissioning project	M. Nakahara (IAEA)
51 • Two-step decontamination strategy development for intermediate level Zr-alloy waste	H. Jungho (Ulsan National Institute of Science and Technology)
81 • A modification of bismuth-based metal organic framework to improve effectiveness in radioiodine capture and feasibility in conversion as a precursor of post-capture process	Y. Jung (KAIST)
65 • Creation of Smart PDMS Sponge for Selective Recovery of Molybdenum from Radioactive Wastes	Y. Zhang (Tokyo Institute of Technology)
21 • Development of Decomposition Technique for Dibutyl Phosphate in Alkaline Liquid Waste	T. Horimai (Japan Nuclear Fuel Limited)
47 • Electrochemical rare earth recovery by Si cathode for a volume reduction of the salt waste form in pyroprocessing	T. Murakami (Central Research Institute of Electric Power Industry)

⌚ 16.00-16.30

Coffee break

⌚ 16.30-18.30

II Rimbaud

2b - FUEL CYCLE OPTIONS INCLUDING TECHNICAL-ECONOMICAL EVALUATIONS - Advanced fuel cycle options for thermal neutron reactor systems

> Chairs: Y. Sagayama (IAEA), G. Grassi (OECD-NEA)

168 • Multirecycling of Plutonium in LWRs R&D Programme in France, A Reactor/Fuel Cycle System Project	C. Evans (Orano)
144 • Advances on Plutonium Multirecycling on PWRs (MIX and MOX)	Y. Rugama (Framatome)
146 • Multirecycling Plutonium on PWR	R. Spaggiari (Framatome)
57 • Comparison of optimized options for the multi-recycling of Reprocessed Uranium in future PWR reactors	F. Laugier (EDF)
161 • Stabilizing Pu inventory using multi-recycling strategies in Pressurized Water Reactors	S. Mirotta (CEA)
164 • Analysis of multi-recycling strategies minimising natural resource and waste impact with DANESS 1g	L. Van Den Durpel (Nuclear-21)

⌚ 16.30-18.30

II Diderot

5e - FUEL RECYCLING - Partitioning and transmutation: fuels and recycling processes

> Chair: G. Lumetta (PNL)

108 • Research and development on minor actinides separation process using CHON extractants in JAEA	T. Matsumura (IAEA)
186 • Demonstration of the i-SANEX process using process levels of 241Am and a CHON stripping agent in a lab-scale centrifugal contactor cascade	D. Whittaker (NNL)
190 • Effects of Aqueous Phase Complexants on Lanthanide and Actinide Oxalate Solubilities	H. Colledge (NNL)
157 • Calculation with PLEIADES-GERMINAL fuel performance code of the SUPERFACT irradiation with updated irradiation conditions	F. Marconi (CEA)
35 • Minor actinides recovery from high level liquid waste using extraction chromatography	M. Takeuchi (IAEA)

⌚ 16.30-18.30

II Colbert

6b - WASTE MANAGEMENT - Treatment / Conditioning

> Chair: C. Ferry (CEA)

10 • Fundamental Study on Vitrification of High-Level Liquid Waste	I. Ishio (Japan Nuclear Fuel Limited)
139 • Cold crucible (...)	E. Sauvage (CEA)
149 • Effect of Introduction of Simultaneous Adsorption System of Mo and PGMs from HLLW on Vitrification Process	K. Takeshita (Tokyo Institute of Technology)
100 • Innovative technique for improving the compatibility of radioactive waste with the blocking matrix	P. Chantereau (NUVIA SAS)
9 • DEM&MELT In-Can Thermal Treatment Solution for Waste Coming from Decommissioning and Dismantling Operations	R. Didierlaurent (Orano)

⌚ 18.45

Royale Room Keynote speech | Philippe Costes (WNA)

⌚ 19.30

Conference Reception | ★ Sponsored by ORANO

JULY, FRIDAY 8

8.00	Registration • Congress Center Main Entrance	
08.30-10.30	Rimbaud	
	1a, 1b - NUCLEAR ENERGY: OUTLOOK & PROSPECTS	
	> Chair : S. Arndt (ANS), F. Sudreau (CEA)	
	120 • Analysis of the relevance of deploying Small Modular Reactors farms to stabilize electrical grids with high share of variable energy sources	C. Boudot (CEA)
	166 • Intra-nuclear sustainability strategies in light of tomorrow's nuclear future: Nuclear-21's 2022 Scenario Analysis	L. Van Den Durpel (Nuclear-21)
	17 • Cradle-to-grave Environmental Footprint Analysis of Energy Sources using Cyclus - from Raw Material Extraction to Waste Generation	D. Ermakova (University of California Berkeley)
	78 • Impact of a possible large scale development of SMRs on natural uranium consumption and deployment of nuclear power worldwide	A. Clerjon (CEA)
	197 • Towards a commercial LFR fleet operating on nitride fuel in Sweden	J. Wallenius (KTH Royal Institute of Technology)
08.30-10.30	Diderot	
	2c - FUEL CYCLE OPTIONS INCLUDING TECHNICAL-ECONOMICAL EVALUATIONS - Focus on MSRs as a fuel cycle option	
	> Chairs: C.Chabert (CEA), F. Laugier (EDF)	
	97 • Feasibility study on chloride salt fast breeder reactor using spent fuel from light water reactor	S. Matsumae (Tokyo City University Setagaya Campus)
	123 • Progress in Chemistry for the Development of the Seaborg Technologies CMSR	J. Amphlett (Seaborg technologies)
	207 • Comparison of chloride and fluoride molten salt reactor concepts for pyrochemical treatments and salt redox control	S. Delpech (CNRS)
	62 • Innovative minor actinides transmutation strategies using Molten Salt Reactors	J. Martinet (CEA)
	126 • Parametric Study of Two-Zone Breed and Burn Molten Chlorides Fast Reactor	J. Krepel (Paul Scherrer Institute)
	34 • Estimation of the vitrified canister production for PWR fleets integrating MSRs with different fuel cycle strategies	L. Tillard (Orano)
	116 • Analysis of value proposition and economic advantages of scenarios based on the use of MSRs as "waste burners"	Y. Meridiano (Orano)
08.30-10.30	Royale Room	
	SP2	
	3a - NUCLEAR FUEL CYCLE FRONT-END - Front-End markets: updates and prospects	
	> Chair: J.-M. Guiheux (Orano)	
	217 • Uranium Resources, Production, Supply and Demand: An IAEA Global Overview	B. Moldovan (AIEA)
	150 • Opportunities and Challenges in the Global Uranium Market	D. Doerksen (Cameco)
	213 • A renaissance is emerging in U.S. ISR uranium mining	A. Adnani (Uranium Energy)
08.30-10.30	Colbert	
	6c - WASTE MANAGEMENT - Disposal / Storage	
	> Chair: V. Wasselin (ANDRA)	
	115 • HLW Interim storage as a key equipment for waste management and recycling strategy	V. Piovesan (Orano)
	183 • HTGR Fuel Storage and Disposal: The Known Unknowns and How to Plug the Knowledge Gaps	L. Kissick (National Nuclear Laboratory)
	18 • Safety assessment for the Rokkasho LLW disposal facilities	R. Hojo (Japan Nuclear Fuel Limited)
	226 • Increase the authorized Very-Low Level Waste disposal capacity of Cires: a project for 2022	F. Gérard (ANDRA)
	110 • Determination of the leaching and speciation of carbon-14 in long-term behaviour of irradiated graphite	J. Comte (CEA)
	69 • Terrestrial Radio-isotopic Thermal Generator (RTG) A solution to valorise isotopes	E. Fourcy (Orano)
10.30-11.00	Coffee break	
11.00-12.30	Royale Room	
	Plenary 4	
	> Moderator : C. Evans (Orano)	
	SMR-Emerging reactor System concepts: development and deployment perspectives of the different technologies highlighting fuel cycle requirements and fuel cycle constraints	
	S. De Groot (Horizon), K. Kallemets (FERMI), C. Levesque (TerraPower), P. Pappano (X-energy/TRISO-X), P. Reid (Moltex), J. Wileman (GE)	
12.30-12.45	Closing: Guillaume Dureau (Orano)	
12.45-13.30	Lunch - END OF THE CONFERENCE	
12.30-16.30	Technical visit*	

*Technical visit

* Visit of Industrial facility for grouping, storage and disposal (Cires). Disposal of very low-level waste (VLLW) since 2003. Collecting radioactive waste produced by activities other than nuclear power production. Temporary storage of radioactive waste from sources other than nuclear power production, pending the development of suitable disposal solutions. Sorting and processing certain waste items.

- 12:30 pm: Departure from Reims Congress Center
- 2:30 pm: Visit of Industrial facility for grouping, storage and disposal (Cires).
- 4:30 pm: Departure for Troyes Train Station

**Posters

46 Nuclear Core Design of a Cartridge type LFR based SMR	M. H. Kim (Kyung Hee University)
163 NESSAT v6.0 : the integrated nuclear energy systems strategies assessment toolbox	L. Van Den Durel (Nuclear-21)
41 Improving the nuclear fuel cycle performance of fast reactors based on a revisited reactor design	J. Sarr (CEA)
66 Modelling the VVER-1000 in OpenMC for coupling with RELAP5	F. Frieb (University of Natural Resources and Life Sciences)
177 A closed fuel cycle option using the MSFR concept with chloride salts and U/Pu cycle	E. Merle (LPSC/IN2P3/CNRS, UGA, G-INP)
24 New UOX/MOX hybrid assembly (CORAIL) for Plutonium multi-recycling in PWR	C. Daletto (CEA)
95 Conversion and enrichment know-how supporting non-nuclear isotope separation	L. Bigot (Orano Chemistry Enrichment)
229 R72 – Cask for spent fuel rods – B(M)F TYPE	J. Thomas (Robatec Industries)
26 Severe Accident Measures for Chemical Facilities in Rokkasho Reprocessing Plant	Y. Kato (Japan Nuclear Fuel Limited)
44 Hybrid process combining solvent extraction / low pressure loss extraction chromatography for a reasonable MA(III) recovery process	Y. Sano (JAEA)
76 A spent fuel characterization study with a Differential Die-Away system using MCNP simulation and a machine learning technique	S. Seol (Hanyang University)
45 Anodic dissolution behaviors of Inconel 600 depending on Ni and Fe additives in chloride salts at 500 °C	Y. Jeon (Ulsan National Institute of Science and Technology)
202 Additional Actinide Separation to Minimize Human Intrusion Radiological Risk to Pyroprocessing Waste	G. Seo (UNIST)
88 Characterization and dissolution of (Ru, Rh, Pd, Mo)-bearing (U, Th)O ₂ heterogeneous mixed oxides for spent fuel modeling	C. Hours (CEA)
72 Innovation in fission products behaviour studies: synthesis and characterization of U _{1-x} Pu _y O _{2-x} SIMfuel	R. Caprani (CEA)
147 Radioactive sludge waste management: calcination process	C. Gobaut (Orano)
145 Waste drying results of the In-Drum Drying	C. Gobaut (Orano)
138 In-Can Vitrification of ALPS Slurries from Fukushima Effluent Treatment Waste using DEM&MELT technology	C. Michel (CEA)
122 Rip & Ship Approach for Disposition of Retired Steam Generators	C. Maufrais (Cyclife groupe EDF)
104 Sorbmatech®Cs, an Innovative Highly Selective Cs Sorbent: First Decontamination of Industrial Wastewater	N. Segond (Orano)
79 Experimental assessment on chemical durability of cementitious final waste-forms containing novel radioactive waste of B-10 enriched boric acid	M. Rostami (Eötvös Loránd University)
75 Direct conditioning feasibility of radioactive organic solvents in a geopolymers binder	O. Dugne (CEA)
114 Study on Solidification Alternatives for Secondary Wastes from Contaminated Water Treatment System in Fukushima Daiichi NPS	T. Koyama (CRIEPI)